

DAFTAR PUSTAKA

- [1] John N. Lillington. *Light Water Reactor Safety: The Development of Advance Models and Codes for Light Water Reactor Safety Analysis*, ELSEVIER, Amsterdam, 1995.
- [2] S. J. Green dan Gad Hetsroni. "PWR Steam Generators". *International Journal of Multiphase Flow*, 21:1-97, 1995.
- [3] Muhammad Eissa, Tarek Nagla, Ayla Badawi. "Main Steam Line Break Analysis in 2775 MWth PWR Reactor Core". *Arab Journal of Nuclear Sciences and Applications*, 50:186-196, 2017.
- [4] *SAR Ringhals 3*. Dokumen teknis, 1602818/2.0, Skärström Lennart PRR3, 2006.
- [5] F. D'Auria, J. L. G. Moreno, G. M. Galassi, D. Grgic dan A. Spadoni. "Three Mile Island Unit 1 Main Steam Line Break Three Dimensional Neutronics/Thermal-Hydraulics Analysis: Application of Different Coupled Codes". *Nuclear Technology*, 142:1, 2003.
- [6] Andy Sofrany Ekariansyah. "Evaluasi Keselamatan Reaktor Tipe PWR pada Kecelakaan Putusnya Jalur Uap Utama". *Sigma Epsilon*, vol. 15, 2011.
- [7] Thangamani Inbanthan, Vishnu Verma, Ram Kumar Singh dan A. K. Ghosh. "Thermal Stress Analysis of Containment Building in Case of Main Steam Line Break (MSLB)". *Nuclear Engineering and Design*, 239:1660-1672, 2009.
- [8] Han-Sik Jung, Dua-Ho Hong, Jong-In Kim, Ae-Ju Cheong and Kyung Won Lee. *Steam Line Break Analysis Using RELAP5/MOD3.3 for Steam Generator Blowdown Load Assessment*. Laporan Penelitian, U.S. Nuclear Regulatory Commission, Washington DC, 2017.
- [9] Mathias Stalek, Jozsef Banati dan Christophe Demaziere. "Main Steam Line Break Calculations Using A Coupled RELAP5/PARCS Model For The Ringhals-3 Pressurized Water Reactor". *ICONE 16: 16th International Conference on Nuclear Engineering*, hal. 1-8, Orlando, 11-15 Mei 2008.
- [10] Westinghouse Water Reactor Divisions. *The Westinghouse Pressurized Water Reactor Nuclear Power Plant*. Westinghouse Electric Corporation, Pittsburgh, 1984.

- [11] Bengt Pershagen. *Light Water Reactor Safety*. Pergamon Press, Stockholm, 1989.
- [12] Paul B. Abramson. *Guidebook to Light Water Reactor Safety Analysis*. Hemisphere Publishing Corporation, Washington, 1985.
- [13] F. D'Auria dan G. Galassi. "Applications Of SYS TH Codes to Nuclear Reactor Design and Accident Analysis". *Thermal Hydraulics in Water-Cooled Nuclear Reactors*. 951-1097, 2017.
- [14] L. Tong. "Heat Transfer in Water-Cooled Nuclear Reactors". *Nuclear Engineering and Design*, 301-324, 1967.
- [15] Code Manual Vol.1-5 RELAP5. Information System Laboratories, Rockville, 2003.