

## DAFTAR PUSTAKA

- [1] A. Nikitin and L. Andreyev, "Floating nuclear power plants," Bellona, 2011.
- [2] "Akademik Lomonosov," *Nuclear Threat Initiative*, 2019.  
<https://www.nti.org/learn/facilities/942/> (diakses 24 April 2021).
- [3] "FNPP in Operation," *Rosatom*, 2020.  
<https://rosatomnewsletter.com/2020/06/29/fnpp-in-operation/> (diakses 24 April 2021).
- [4] International Atomic Energy Agency, "KLT-40S Overview," 2013.
- [5] International Atomic Energy Agency, *Advances in Small Modular Reactor Technology Developments*. International Atomic Energy Agency, 2020.
- [6] B. Gihm and V. Snell, "Survey of Design and Regulatory Requirements for New Small Reactors," 2014.
- [7] International Atomic Energy Agency, "Fundamental Safety Principles," Vienna, 2006.
- [8] International Atomic Energy Agency, *Safety of Nuclear Power Plants: Design*, no. SSR-2/1. Wina: International Atomic Energy Agency, 2016.
- [9] US Nuclear Regulatory Commission, "Anticipated Transients Without Scram for Light Water Reactors," Washington, D.C., 1978.
- [10] International Atomic Energy Agency, *Anticipated Transients Without Scram for WWER Reactors*, no. IAEA-EBP-WWER-12. Wina, 1999.
- [11] S. Chaudhary, P. K. Kumar, Y. K. Pandey, and G. Biswas, "Thermal-Hydraulic Analysis of Partial Loss of Forced Reactor Coolant Flow with Non-Uniform and Asymmetric Loop Flow Mixing in VVER-1000," in *National Conference on Fluid Mechanics and Fluid Power*, 2018, no. 071.
- [12] B. Foad, S. H. Abdel-latif, and T. Takeda, "Reactivity Feedback Effect on Loss of Flow Accident in PWR," *Nucl. Eng. Technol.*, vol. 50, no. 8, pp.



- 1277–1288, 2018, doi: 10.1016/j.net.2018.07.012.
- [13] M. F. Alkodri, *Analisis Parameter Termal Hidraulika Sistem Primer Tertutup Reaktor KLT-40S saat Kondisi Transien Coastdown Pompa Pendingin Reaktor*, Skripsi, Departemen Teknik Nuklir dan Teknik Fisika, Universitas Gadjah Mada, Yogyakarta, 2020.
- [14] A. Kristanto, *Pemodelan dan Analisis Termal Hidraulika Parameter – Parameter Operasi Teras Reaktor KLT-40S pada Kondisi Stabil (Steady-State) dan Transien Menggunakan RELAP5-3D*, Skripsi, Departemen Teknik Nuklir dan Teknik Fisika, Universitas Gadjah Mada, 2017.
- [15] A. C. Lesmana, *Pemodelan Neutronik-Termalhidrolik untuk Kondisi Ajek dan Transien Tekanan serta Laju Aliran Massa Inlet pada Reaktor KLT-40S Menggunakan RELAP5-3D*, Skripsi, Departemen Teknik Nuklir dan Teknik Fisika, Universitas Gadjah Mada, 2019.
- [16] T. Simiao *et al.*, “Thermal-Hydraulic Analysis of the EBR-II SHRT-45R Unprotected Loss of Flow Experiment with Modified RELAP5,” in *International Conference on Nuclear Engineering*, 2017, pp. 1–7.
- [17] S. Abalkhail, Y. Lee, and K. Hwan Bae, “Evaluation of Anticipated Transient Without Scram Initiated by Total Loss of Reactor Coolant Flow,” *Trans. Korean Nucl. Soc. Autumn Meet.*, 2018.
- [18] A. L. Costa, P. A. de L. Reis, C. Pereira, M. A. F. Veloso, and C. A. M. da Silva, “Research Reactor Analysis Using Thermal Hydraulic and Neutron Kinetic Coupling,” in *International Conference on Nuclear Engineering*, 2014, pp. 1–5.
- [19] A. V. Varentsov, D. N. Solntsev, V. D. Sorokin, M. A. Legchanov, and A. E. Khrobostov, “Experimental Researches of Mass Exchange of Heat-Carrier Flow In Fuel Assemblies of the KLT–40S and RITM–200 Reactors,” *Proc. Nizhny Novgorod State Tech. Univ. R. E. Alekseeva.*, no. 5, pp. 47–51, 2014.
- [20] V. Beliaev and V. Polunichev, “Basic Safety Principles of KLT-40C Reactor



Plants,” Nizhny Novgorod, 2000.

- [21] J. J. Duderstadt and L. J. Hamilton, *Nuclear Reactor Analysis*. Ann Arbor: John Wiley & Sons, Inc, 1976.
- [22] W. M. Stacey, *Nuclear Reactor Physics: Second Edition*. Atlanta, 2007.
- [23] J. R. Lamarsh, “Introduction to Nuclear Engineering, 2nd Edition,” *Addison-Wesley Publishing Company*. Addison-Wesley Publishing Company, Inc., Larchmont, 1983.
- [24] N. E. Todreas and M. S. Kazimi, *Nuclear Systems I: Thermal Hydraulic Fundamentals. Vol. 1*. 1990.
- [25] International Atomic Energy Agency, “Considerations on the Application of the IAEA Safety Requirements for the Design of Nuclear Power Plants,” Vienna, 2016.
- [26] International Atomic Energy Agency, “Accident Analysis for Nuclear Power Plants with Pressurized Water Reactors,” Vienna, 2003.
- [27] International Atomic Energy Agency, “Guidelines for Accident Analysis of WWER Nuclear Power Plants,” Vienna, 1995.
- [28] Idaho National Laboratory, “RELAP5-3D Code Manual: Code Structure, System Models and Solution Methods,” 2015.
- [29] M. K. Ahimsa, *Analisis Desain Fuel Assembly dan Teras Reaktor PLTN Terapung KLT-40S Menggunakan Kode SERPENT*, Skripsi, Departemen Teknik Nuklir dan Teknik Fisika, Universitas Gadjah Mada, 2017.
- [30] B. Zohuri, *Thermal-Hydraulic Analysis of Nuclear Reactors*. Albuquerque, 2016.
- [31] J. March-Leuba, “Density-Wave Instabilities in Boiling Water Reactor,” Oak Ridge, 1992.

